FINAL BASEWIDE RADIOLOGICAL REMOVAL ACTION ACTION MEMORANDUM HUNTERS POINT SHIPYARD SAN FRANCISCO, CALIFORNIA

November 19, 2001

DEPARTMENT OF THE NAVY
Southwest Division
Naval Facilities Engineering Command
San Diego, California

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ACRONYMS AND ABBREVIATIONS

§ Section

AM Action Memorandum

ARAR applicable or relevant and appropriate requirement

BRAC Base Realignment and Closure

Ca-HSC California Health and Safety Code

CERCLA Comprehensive Environmental Response, Compensation, and Liability Act

CFR Code of Federal Regulations

DHS California Department of Health Services

DTSC California Environmental Protection Agency Department of Toxic Substances Control

EPA U.S. Environmental Protection Agency

HPS Hunters Point Shipyard

HRA Historical Radiological Assessment

IR Installation Restoration

MARSSIM Multi-Agency Radiation Survey and Site Investigation Manual

Navy U.S. Department of the Navy

NCP National Oil and Hazardous Substances Pollution Contingency Plan

NPL National Priorities List

NRC U.S. Nuclear Regulatory Commission NRDL Naval Radiological Defense Laboratory

O&M Operation and maintenance

PRC PRC Environmental Management, Inc.

PRG Preliminary remediation goal
PRP Potentially responsible party

RI/FS Remedial investigation and feasibility study

RWQCB California Regional Water Quality Control Board, San Francisco Bay Region

SARA Superfund Amendments and Reauthorization Act of 1986

TCRA Time-critical removal action

TtEMI Tetra Tech EM, Inc.

USC United States Code

Hunters Point Shipyard San Francisco, California 94124

November 19, 2001

Subject: Action Memorandum for Time-Critical Removal Action of Radiological

Materials in Soils, Debris, or Structures at Hunters Point Shipyard,

San Francisco, California

Site Status: National Priorities List: listed in November 1989;

Parcel A was transferred in December 2004

Removal Category: Time-Critical Removal Action

CERCLIS ID: CA1170090087

Site ID: 0902722

I. PURPOSE

The purpose of this action memorandum (AM) is to document for the administrative record the U.S. Department of Navy's (Navy) decision to undertake time-critical removal actions (TCRA) at areas throughout the base that may contain localized radiological contamination in soils, debris/slag, and buildings at Hunters Point Shipyard (HPS), as identified in the *Historical Radiological Assessment, Volume II, Use of General Radioactive Materials, 1939-2003, Hunters Point Shipyard* (HRA). The Department of Defense has the authority to undertake Comprehensive Environmental Response, Compensation, and Liability Act (CERCLA) response actions, including removal actions, under Title 42 of the *United States* Code (USC) Section (§) 2705 and the federal Executive Order 12580. Further, this removal action is consistent, to the maximum extent possible, with Chapter 6.8 of California Health and Safety Code (Ca-HSC).

The proposed removal actions described in this AM will substantially eliminate identified pathways of exposure to hazardous substances for surrounding populations and nearby ecosystems, such as nearby wetlands and the San Francisco Bay. Removal actions performed per this AM are anticipated to be complete cleanups to, or below, the cleanup goals specified in this document.

Removal actions performed per this AM are deemed consistent with (1) the factors set forth within the National Oil and Hazardous Substances Pollution Contingency Plan (NCP) Title 40 of the *Code of Federal Regulations* (CFR) Part 300, and (2) Chapter 6.8, Ca-HSC, based on the findings below.

Threats to public health or welfare:

- Nearby human populations may be affected by exposure to low-level radioactive materials
- Low-level radioactive materials may migrate or be released because of their presence near the surface

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• Low-level radioactive materials may migrate or be released because of weather conditions

Threats to the environment:

- Nearby animals, and food chains may be affected by exposure to low-level radioactive materials
- Radioactive materials can have very long half-lives. Their release into the environment could be detrimental

No nationally significant or precedent setting issues exist for this site.

SITE CONDITIONS AND BACKGROUND II.

This section describes the site history and background of HPS, summarizes each action conducted to date, and presents the findings of previous characterizations of radioactivity at HPS.

SITE DESCRIPTION A.

The following sections summarize characteristics of the site, any releases or threatened releases of contaminants, and the status of the site on the National Priorities List (NPL).

1. **Removal Site Evaluation**

To date, several radiological site investigations have been conducted at HPS to assess the presence of radioactive materials remaining from past operations associated with the Navy Radiological Defense Laboratory (NRDL) and ship decontamination and maintenance procedures. Those investigations delineated certain areas at which low-level radioactive contaminants were found, and some of those areas have been addressed under a previous radiological removal action.

As investigations continue, additional areas throughout HPS are being considered for their potential to contain low-level radiological contamination. This AM addresses those potential areas through removal and off-site disposal actions.

Three general types of media exist in which radiological contamination may be found:

- 1. Soils
- 2. Debris/slag
- 3. Buildings: walls, foundations, slabs, and so on

Examples of previously identified low-level radiological contamination include anomalies found in soils near buildings; debris/slag containing embedded radium dials; and surface contamination on concrete slabs, walls, and piping associated with buildings.

2. **Physical Location**

HPS is located in the City and County of San Francisco, California, and is shown on Figure 1. HPS is situated on a long promontory in southeast San Francisco, extending eastward into San Francisco Bay. The primary mission of HPS was naval shipyard activities. HPS consists of 936 acres, 494 of which are on land, and is divided into six parcels (B, C, D, E, E-2, and F) to facilitate environmental investigation and cleanup activities. Parcel A was conveyed to the City of San Francisco in December, 2004.

The climate is characterized as temperate, or Mediterranean, which typically has moist mild winters and dry summers. The average annual precipitation in the area is 21.79 inches. The precipitation occurs

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mostly during the months of December, January, and February. There are public residencies within a mile radius of HPS and the nearest major thoroughfare is I-280, located roughly five miles west of the site.

3. Site Characteristics

HPS is a federally owned facility, with began using radioactive materials in the 1940s with the formation of the NRDL. In 1969, radiological studies at HPS ended, and NRDL buildings were decontaminated and cleared for unrestricted reuse. The Navy also conducted ship decontamination, repair, and dismantling activities, which generated radium dial and sandblast grit waste streams. During NRDL operations, the Navy utilized a radiological waste-handling program, which included removal of high-level radioactive materials from HPS and transport of the materials to an off-site disposal area.

As a result of past operations, some NRDL-associated buildings have been found to contain low-level radiological contaminants and radium dials have been found embedded in both debris and slag, or buried in disposal areas. Hazardous materials have also been found at HPS. The site was placed on the NPL in 1989, pursuant to CERCLA, as amended by the Superfund Amendments and Reauthorization Act of 1986.

In 1991, HPS was slated for closure pursuant to the terms of the Defense Base Closure and Realignment Act of 1990 (Public Law 101-510). Closure of HPS includes conducting environmental remediation activities and transfer of the property to the City of San Francisco for future non-defense reuse.

4. Release or Threatened Release into the Environment of a Hazardous Substance or Pollutant or Contaminant

The radioisotopes encountered to date, and likely to be encountered during future investigations, include americium-241, cobalt-60, strontium-90, cesium-137 (and daughter products), europium-154, radium-226, thorium-228, thorium-232, and uranium-235 (and daughter products), and are hazardous substances, as defined by §101(14) of CERCLA, and pollutants or contaminants, as defined by §101(33) of CERCLA.

Because of the presence of low-level radioactive materials in areas exposed to erosion and weathering, a threat of migration and release to surrounding populations and the environment could exist.

To date almost all radioactive materials encountered at HPS have been isolated from human contact and located in restricted-access areas. However, the threat of release does exist because of the persistence of radiological materials, their presence in areas designated for future unrestricted use, and their presence in areas that may be affected by weather and erosion.

Removal actions conducted in accordance with this AM are therefore designed to (1) substantially reduce ionizing radiation to cleanup goals and (2) eliminate identified pathways of exposure to ionizing radiation.

5. National Priorities List Status

HPS was placed on the NPL on November 21, 1989, with a Hazard Ranking Score of 48.77. Parcel A was delisted in February 1999 and transferred in December 2004. Each parcel has undergone, or is undergoing, a CERCLA remedial investigation and feasibility study (RI/FS); RI/FSs have been completed for Parcels A and B, and are being conducted for Parcels C, D, E and E-2.

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6. Maps, Pictures, and Other Geographical Representations

Figure 1 shows the location of HPS, and Figure 2 presents all currently known radiological-related areas at HPS. Many radiological areas have already been addressed by previous investigations and by a previous radiological removal action.

B. OTHER ACTIONS TO DATE

Several radiological investigations and radiological removal actions have been conducted at HPS. The following sections summarize those actions.

1. Previous Actions

Four phases of radiological investigations were performed at HPS, beginning in 1991. Phases I and II delineated the surface and subsurface distribution of radium-containing devices. Phases III and IV recommended and performed the removal of anomalies near Buildings 364, 509, 529, and 707 in Parcels D and E. Phase V conducted radiological surveys and remedial actions in Parcels B, C, D, and E. Each investigation is summarized below.

1.1 Phase I

The phase I radiological investigation was conducted in 1991 to evaluate the extent of radium-containing devices identified in a surface radiation survey conducted by Harding Lawson Associates (1990) in 1988. The phase I investigation included Installation Restoration (IR) sites 01/21, 02, 03, and portions of IR site 11/14/15.

Over 300 radium-containing point sources were detected in a centralized area in IR-02 Northwest during the phase I investigation, and additional anomalies were observed in IR-01/21 and IR-02 Southeast. A dial with anomalously high gamma activity was also found on the door of a combination safe in IR-11/14/15.

Thirteen soil samples collected from the disposal area in IR-02 Northwest contained radium-226 at concentrations exceeding background levels. One soil sample collected from IR-01/21 and two soil samples collected from IR-02 Southeast contained radium-226 at concentrations exceeding background levels (PRC Environmental Management, Inc. [PRC] 1992).

The phase I investigation concluded that elevated gamma activity was a result of the presence of radium containing devices in surface soil at scattered locations at IR-01/21 and on the surface and in the subsurface of the centralized disposal area in IR-02 Northwest, which extends into IR-02 Central.

The phase I radiation investigation recommended an investigation of the subsurface distribution of radium-containing devices in soil in IR-02 Northwest, removal of the combination safe from IR-11/14/15, and further speciation of radiological analytes in groundwater. The first and second recommendations have been performed, and the third is ongoing.

1.2 Phase II

The phase II radiological investigation was conducted in 1993 to delineate the subsurface distribution of radium-containing devices in the IR-01/21 landfill and in the disposal areas in IR-02 Northwest and IR-02 Central. Field activities included excavation of trenches and test pits, collection of soil samples, and collection of air samples (PRC 1996).

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Excavation activities at the disposal area in IR-02 Northwest and in IR-02 Central revealed 111 discrete subsurface gamma-emitting point sources, all located within a well-defined disposal area. A large amount of industrial and construction debris was also found mixed with soils in the disposal area. Radium-containing devices and industrial debris were detected at the surface in IR-01/21, but not in the subsurface of IR-01/21 or at the beach and intertidal areas of IR-02 Northwest (PRC 1996).

The phase II radiological investigation concluded that the disposal area in IR-02 Northwest and IR-02 Central was the primary disposal area for all radium-containing devices generated at HPS as a result of ship repair and maintenance activities, and that radium-containing devices were only present on the surface of the landfill in IR-01/21.

1.3 Phase III

The phase III radiological investigation was conducted in 1997 to address concerns about the use, storage, and disposal of radioactive materials during past NRDL operations at HPS. The goal of the phase III investigation was the eventual release for unrestricted use, all remaining buildings and sites not previously released, including three formerly used defense sites. Radiological surveys were conducted within and around Buildings 506, 509, 517, and 529.

The Phase III radiological investigation recommended the following actions (Tetra Tech EM Inc. [TtEMI] 1997):

- Excavation of a potential buried point source behind Building 529
- Excavation of an area with an anomalous count rate of 9,374 counts per minute near Building 509
- Further study of Buildings 364 and 707 (TtEMI 1997)

All phase III recommendations were implemented in the phase IV investigation, or in the removal action.

1.4 Phase IV

The phase IV radiological investigation was conducted in 1999 to quantify ambient concentrations of specific radionuclides and to further characterize two radiological sites located near Buildings 364 and 707. The goal of the phase IV investigation was free release for industrial use of the areas located near Buildings 364 and 707.

The phase IV investigation recommended the following actions:

- Removal of a cesium-137 spill site near Building 364
- Removal of anomalies near the former locations of Buildings 509, 529, and 707

Both phase IV recommendations were implemented in the remedial action, which began in February 2001.

1.5 Interim Investigations Between Phase IV and Phase V

Three interim investigations were conducted after the Phase IV radiological investigation. Available information for each investigation is summarized below:

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1.5.1 1999 October IT Corporation Investigation

Sandblast waste was discovered and removed from an excavation site at IR-07. Five samples were collected and analyzed for radioactivity. Results were indicative of background concentrations.

1.5.2 2001 TtEMI Investigation

In June and July 2001, TtEMI contracted a survey of the Gun Mole Pier (Regunning Pier). Gamma and beta measurements were obtained on the pier to determine whether elevated radioactivity levels remained from previous operations. The measurement points were based on a newly found drawing, indicating the previous location of the concrete test pad and NRDL barge on the pier. Gamma levels were measured using a portable detection instrument equipped with a NaI scintillation probe. The measurements were made both in a systematic grid pattern and biased locations specifically over drains, surface cracks, and other unusual features.

Surface radioactivity at selected locations was also measured using a Geiger-Mueller detector sensitive to beta radiations.

Findings indicated that only background levels of radioactivity were present in the areas surveyed.

1.5.3 NWT Interim Investigation and Removal Action

During 2001, New World technology (NWT) performed a removal action at the tank vault behind Building 364. Others had removed the tanks, piping, and support equipment previously, and the remaining vault surfaces had been identified as exceeding site release criteria. The concrete vault was broken and removed using standard industrial demolition equipment, packaged, and shipped for disposal to a licensed disposal facility. Surveys and soil sampling performed following removal of the concrete vault indicated that no residual contamination remained that exceeded site release criteria. The excavation was backfilled.

Additionally, a Characterization Survey of the Parcel E shoreline was performed. Gamma scans were conducted over pre-positioned grids using 2-by-2-inch NaI detectors and Ludlum Model 2360 Data Logger instruments. The shoreline survey encompassed areas within approximately 50 feet of the mean tide line with each grid assigned an individual identifier. The grid corners were identified using global positioning system coordinates. Several areas were noted during the survey that exceeded background gamma radiation levels, most significantly the areas known as the "Metal Debris Reef" and "Metal Slag Area." Samples obtained from those locations identified Ra-226 as the contaminant. The elevated reading locations were noted on survey maps. No removal actions were taken at the time of the survey.

1.6 Phase IV Removal Action

Based on the results of the phase IV investigation, a removal action was begun in early 2001 and has been completed. The removal action was designed to focus on Buildings 364, 509, 529, and 707, as identified in the final AM dated August 2000 (TtEMI 2000a).

The initial goals of the removal action were to remove (1) radioactive anomalies found in the upper soil layer at levels exceeding U.S. Environmental Protection Agency (EPA) decay-corrected preliminary remediation goals (PRG) and (2) a subterranean concrete sump near Building 364. The sump was previously used to store liquids with radiological contamination. Those goals were met, and the scope of the removal action was expanded to address additional contamination found in concrete materials near Building 364 and to conduct a shoreline radiological survey focusing on characterization of intertidal debris.

1.7 Phase V Investigations

Beginning in January 2002, NWT conducted scoping and Characterization Surveys, soil and other media sampling programs, remediations, and Final Status Surveys at various areas and in various buildings at HPS in accordance with Multi-Agency Radiation Site Survey and Investigation Manual (MARSSIM) guidelines. The investigations and surveys were in support of the release of buildings or areas that had been identified as areas where radioactive materials had been used or areas where remedial actions to remove known contamination had occurred.

The phase V Investigations were conducted within a standard protocol that allowed for application of MARSSIM guidelines in the survey process. Each site was assessed for potential radionuclides of concern with surveys designed according to the MARSSIM area classification (Class 1, 2, or 3). In general, the surveys included gamma scans, gamma static readings, alpha/beta static readings, dose rate measurements, alpha/beta swipes, H-3 swipes (if appropriate), and sample analysis (alpha or gamma spectroscopy or beta analysis, as appropriate). The extent of the surveys depended upon the classification of the area. Class 1 surveys covered 100 percent of the area, Class 2 surveys covered 50 percent of the area, and Class 3 surveys covered 20 percent of the area. Static measurements were distributed accordingly.

If contamination was found in a Class 3 area, a 100 percent Characterization Survey was conducted followed by remediation as appropriate. A Class 1 Final Status Survey followed these actions.

1.8 Phase V Removal Action

Based on the results of the phase V investigation, a removal action took place in 2002 and 2003. The removal action was designed to focus on Buildings 351A, Shack 79, and the Former NRDL Site.

All radioactive materials removed as part of these removal actions were properly stored, transported, and disposed at an approved off-site facility. Post excavation confirmation samples were collected from soil excavation sites to ensure complete removal and achievement of the cleanup goals. All successfully excavated areas were backfilled with clean materials. Some remedial action remains at Shack 79.

2. Current Actions

Current radiological actions at HPS include removal actions that were initiated under this AM and implementation of recommended actions identified in the HRA. These actions are described below.

2.1 TCRAs

Three TCRAs are currently being conducted on site. These removal actions are being conducted in accordance with the requirements of the CERCLA and the NCP. Since these removal actions are being performed in accordance with Section 121(e) of the CERCLA and to support a TCRA, no local permits are necessary. All substantive requirements are being met.

Metal Debris Reef / Metal Slag Area

The TCRA at the Metal Debris Reef and Metal Slag Areas involves the removal of radioactively contaminated metal debris and slag within Parcels E and E-2 at HPS. (Why not just clearly state that the MDR is in parcel E and the MSA is in parcel E-2?)

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The removal action is proceeding in accordance with the *Final Removal Action Design and Implementation Work Plan* [Tetra Tech EC, Inc. (TtEC), 2005] for this TCRA and includes radiological surveying and remediation, excavation of metal slag and debris, site restoration, and waste disposal.

PCB Hot Spots

The TCRA for the Polychlorinated Biphenyl (PCB) Hot Spot Soil Excavation Site (PCB Hot Spot) involves the removal of soils containing PCB, total petroleum hydrocarbons (TPH), and any radioactive contaminants.

The removal action is proceeding in accordance with the *Final Removal Action Design and Implementation Work Plan* (TtEC, 2005) for this TCRA and includes radiological surveying and remediation, excavation of PCB- and TPH-contaminated soil, site restoration, and waste disposal.

IR-02 Northwest and Central

This TCRA is for the extraction of debris and soil containing radioactive contaminants that may be present in the IR-02 Northwest and Central area.

The removal action is proceeding in accordance with the *Final Removal Action Design and Implementation Work Plan* (TtEC, 2005) for this TCRA and includes radiological surveying and remediation, excavation of soil and debris, site restoration, and waste disposal.

2.2.1 Radiologically Impacted Site Surveys

Building 322

This survey was performed to support release of the building for demolition and disposal. After the building was demolished, a Final Status Survey was designed as a MARSSIM (NUREG-1575) Class 1 and Class 2 survey to release the site for unrestricted reuse.

Building 819

This survey was performed to determine if residual radioactivity was present at the site. The survey was designed as a MARSSIM (NUREG-1575) Class 1 survey so that if no contamination was found above the release criteria, the data would be used to support the Final Status Survey.

Building 114 Site

The Building 114 Site is the former location of the demolished Building 114. This survey is being performed to determine if residual radioactivity is present at the site. The survey has been designed as a MARSSIM (NUREG-1575) Class 1 and Class 2 survey so that if no contamination is found above the release criteria, the data can be used to support the Final Status Survey.

Building 146

This survey is being performed to determine if residual radioactivity is present at Building 146. The surveys have been designed as MARSSIM (NUREG-1575) Class 1 and Class 2 surveys. This methodology will allow the use of the survey data to support the Final Status Survey if no contamination is found to exceed the release criteria.

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2.3 Historical Radiological Assessment

The HRA was conducted to evaluate all previous uses of radioactive materials at HPS and to assess their potential to impact the site. The final version of the HRA was issued in August 2004. Based on the recommendations of the HRA, a total of 84 HPS sites have been designated as "impacted." This indicates that the site has a potential for radioactive contamination based on historic information or is known to contain radioactive contamination. These impacted sites, broken out by parcel, include:

- Parcel B 14 sites
- Parcel C 12 sites
- Parcel D 19 sites
- Parcel E -33 sites
- Parcel F − 2 sites
- Off-Base Facilities 1 site
- Base-Wide Areas 3 sites

C. STATE AND LOCAL AUTHORITIES ROLE

The California Department of Health Services (DHS) and EPA have actively participated in the radiological investigations and the radiological removal actions at HPS. In the past, EPA has provided site-specific input for the establishment of removal action cleanup goals and investigative strategies.

The California Environmental Protection Agency Department of Toxic Substances Control (DTSC) and California Regional Water Quality Control Board, San Francisco Bay Region (RWQCB) are also regulatory agency stakeholders.

1. State and Local Actions to Date

As previously discussed, federal Executive Order 12580 delegates to the Department of Defense the President's authority to undertake CERCLA response actions. Congress further outlined this authority in its Defense Environmental Restoration Program Amendments, which can be found at 10 USC §2701-2705. Both CERCLA §120(f) and 10 USC §2705 require Navy facilities to ensure that state and local officials be given timely opportunity to review and comment on Navy response actions. CERCLA §120 further requires the Navy to apply state removal and remedial action law requirements at its facilities.

Accordingly, DHS, DTSC, and RWQCB have provided technical advice and oversight during phases of the RI/FS process, during previous radiological investigations, and during current and future radiological removal actions.

2. Potential for Continued State or Local Response

DHS, RWQCB, and DTSC deferred to EPA for development of cleanup goals for the previous radiological removal action. Those cleanup goals were also chosen for this removal action. DHS, RWQCB, and DTSC will continue to provide input through review of radiological documents and participation in the Base Realignment and Closure (BRAC) Cleanup Team.

It is expected that the Navy's BRAC account funds will continue to be the exclusive source of funding for this program.

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III. THREATS TO PUBLIC HEALTH, WELFARE, OR THE ENVIRONMENT AND STATUTORY AND REGULATORY AUTHORITIES

In accordance with the NCP, the following threats must be considered in determining the appropriateness of a removal action (40 CFR §300.415[b][2]):

- Actual or potential exposure to hazardous substances, pollutants, or contaminants of nearby populations, animals, and food chains
- Actual or potential contamination of drinking water supplies and sensitive ecosystems
- Hazardous substances, pollutants, or contaminants in drums, barrels, tanks, and other bulk storage containers that may pose a threat of release
- High levels of hazardous substances or pollutants or contaminants in soils largely at, or near, the surface that may migrate
- Weather conditions that may cause hazardous substances, pollutants, or contaminants to migrate or to be released
- Threat of fire or explosion
- Other situations or factors that may pose threats to human health or the environment

A. THREATS TO PUBLIC HEALTH OR WELFARE

Three potential threats to public health or welfare exist:

- Nearby human populations may be affected by exposure to low-level radioactive materials
- Low-level radioactive materials may migrate or be released because of their presence near the surface
- Low-level radioactive materials may migrate or be released because of weather conditions

Because of the possible adverse health effects from ionizing radiation (EPA 1998) and the long decay periods (half-lives) for many radionuclides, removal and off-site disposal is considered the most effective option for most of the radiological contaminants found at HPS. Physical removal of radiological materials will ensure that the potential for diffuse radioactivity is reduced to levels that meet or are below cleanup goals.

B. THREATS TO THE ENVIRONMENT

Two potential threats to the environment exist:

- Nearby animals and food chains may be affected by exposure to low-level radioactive materials
- Radioactive materials can have very long half-lives; therefore, their release into the environment could be detrimental

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Physical removal of radiological materials from HPS also provides the most effective option for mitigation of threats to the environment from ionizing radiation.

IV. DETERMINATION OF ENDANGERMENT

Results of radiological investigations conducted to date (TtEMI 1997, 2000b) demonstrate that current conditions at HPS may present immediate and severe threats to the aquatic ecosystem, public health, welfare, or the environment.

Actual or threatened releases of radiological materials from HPS, if not addressed by implementing the response action selected in this AM, may present an imminent and substantial endangerment to public health, welfare, or the environment. The primary endangerment mechanism is through migration: if radiological contaminants migrate, they have the potential to contaminate water and soils. Water and soil contamination could be long lasting, since some radionuclides have half-lives in the tens of thousands of years.

The HRA provides a comprehensive review and assessment of the affect of past radiological operations at HPS.

V. PROPOSED ACTIONS AND ESTIMATED COSTS

The following sections summarize the actions proposed for any TCRA performed per this AM.

A. PROPOSED ACTION

The proposed action for localized radioactive material present at HPS is to physically remove it and dispose of the material at an off-site disposal facility. For purposes of this AM, localized is defined as any area less than approximately 3 acres in which radioactive material is the primary risk driver. The definition for "localized" is based on the results of past radiological investigations, the size of the radium dial disposal area in IR-02 Northwest and IR-02 Central, and the size of the intertidal debris area; these areas are assumed to be the largest probable areas this AM would address.

Estimates on the quantity of radiological materials that will be removed per this AM remain pending, until additional sites are identified for removal actions and more accurate information is gathered. Removal actions performed per this AM will comply with the off-site policy by using a fully permitted off-site disposal facility.

Removal actions performed per this AM are subject to the cleanup goals listed in Table 1, for soils/debris and surfaces, respectively. Before initiating a removal action per this AM, the area being considered will be delineated using real-time radiation detection devices or soil sampling and analyses.

The cleanup goals for radionuclides in soils were used during the previous radiological removal actions, and Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM) guidance is being used to apply the cleanup goals. If additional radionuclides listed in Table 2 are encountered during removal actions, cleanup goals will be derived using regulatory involvement.

Areas where radiological contamination may be too pervasive to conduct a localized removal action, where radiological contamination is not the primary risk driver, or where excavation activities pose a high risk to workers, will be addressed as part of the ongoing CERCLA process. The IR-01/21 landfill is one example of an area that has multiple risk drivers, has a large affected area, and would pose excessive hazards to workers performing excavation activities.

Table 3 provides a listing of radiologically impacted sites at HPS that were identified in the HRA (NAVSEA, 2004). Table 3 does not include those sites that were in the former Parcel A or the Formerly Used Defense Sites (FUDS). Assessments of potentially contaminated media and migration pathways, as

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well as recommended actions are detailed in Table 3. The impacted sites listed in Table 3 will be addressed under this AM.

1. Proposed Action Description

Physical removal and off-site disposal of radiological materials will follow the general steps listed below for three types of environmental media in which radiological contamination is likely to be encountered at HPS. Removal actions will be preceded by preparation of site-specific work plans. To the extent practicable, radioactive materials will be segregated from other materials (such as construction debris or nonaffected soils) to minimize radiological waste stream generation.

Soils

- Delineation of radiological contamination using real-time radiation detection instruments or soil sampling and analyses
- Excavation of radiological materials and proper off-site disposal
- Soil confirmation sampling and analyses; comparison of results against cleanup goals listed in Table 1 using MARSSIM methodology
- Site backfilling and restoration

Debris/Slag

- Delineation of contamination, or general area of suspected contamination, using real-time radiation detection instruments or sampling and analyses
- Collection and segregation of radioactive materials
- Proper off-site disposal of materials
- Comparison of remediated area against cleanup goals listed in Table 1, depending on the surrounding environmental media (soils or surfaces) using MARSSIM methodology
- Site backfilling or restoration

• Concrete Surfaces (walls, slabs, and foundations)

- Delineation of radiological contamination using real-time radiation detection instruments or wipe samples and analyses
- Decontamination of surfaces by acid or solvent washing or mechanical removal such as scabbling (scabbling will be preferred in order to reduce waste stream generation)
- Proper off-site disposal
- Comparison of residual radioactivity to the cleanup goals listed in Table 1, using MARSSIM methodology

Note: if surface decontamination is not technically feasible, the entire structure may be removed and disposed of appropriately.

If radiological areas are found in or near wetlands or intertidal areas, removal actions will be modified to minimize the affect to those areas.

The following laboratory analyses are associated with characterization of the radiological materials that may be addressed by this AM:

- Isotopic Americium and Uranium (234, 235, 238) analyses in soil by American Society for Testing and Materials method D3972-90M
- Gamma spectroscopy analyses in soil by EPA method 901.1

Institutional controls will not be required for removal actions performed under this AM. Radiological areas that cannot be addressed by this AM (for example, areas with large quantities of low-level radiological waste or areas where radiation is not considered the primary risk driver) will continue to be evaluated by the ongoing CERCLA process. Radiological sites not addressed under this AM will continue to have restricted access, until a final remedy is selected.

Postremoval site controls will not be required following removal actions performed per this AM, since the intent of each removal action is to reduce radioactive contaminants to or below the cleanup goals.

2. Contribution to Remedial Performance

Removal of radiological contamination per this AM will allow for the ongoing CERCLA process to address any remaining contamination and will avoid future "mixed waste" (waste with both chemical and radiological contamination). Each removal action taken per this AM will be performed to achieve specified cleanup goals, and will be intended as the final radiological remedy at each site. Removal actions taken per this AM will also take into account the City of San Francisco's reuse plan for the site.

3. Description of Removal Alternative

Several removal action alternatives were considered for use in this AM; however, physical removal and proper off-site disposal was the only viable alternative retained for evaluation.

Other actions such as in-place stabilization, and removal and consolidation in the closed HPS industrial landfill, were considered; however, those alternatives would not physically remove the contamination (requiring long operation and maintenance [O&M] periods), would involve large costs, or would require restricted reuse of certain areas of HPS for long periods of time.

The steps required to remove and properly dispose of low-level radioactive materials at an approved offsite facility were detailed in Section V.A.1.

Removal and proper disposal of radioactive materials will provide a timely response and the best option for protection of human health and the environment. Previous radiological soil removals have been completed within several months, and achievement of cleanup goals ensures that human health risks related to radiological materials are eliminated from the site in question.

The Sections 3.1 and 3.2 summarize the criteria used to evaluate the proposed alternative and results of the evaluation.

3.1 Evaluation Criteria

Three criteria were used to evaluate the removal and disposal alternative proposed in this AM: effectiveness, implementability, and cost.

Effectiveness

Three general factors were considered in evaluating effectiveness: (1) overall protection of human health and the environment, (2) short-term effectiveness, and (3) long-term effectiveness and permanence.

Implementability

This criterion addresses the technical and administrative feasibility of implementing the removal action. Items evaluated include (1) the availability of services and materials required during implementation of the action, (2) the institutional or social concerns that could preclude the action, and (3) state and community concerns that could affect implementation. The following factors were considered:

- Technical feasibility: the ease or difficulty of implementing the alternative and the reliability of the technology
- Administrative feasibility: activities, such as obtaining waivers or permits, requiring coordination with other offices and agencies

Cost

This criterion is concerned with the estimated costs of the alternatives, and is based on previous radiological removal actions for soils and building surfaces. O&M costs were not considered in the cost evaluation since removal actions will be performed in less than a year, and no follow-on costs are associated once this removal action has been completed.

3.2 Evaluation of Proposed Removal and Off-site Disposal Action

The removal and off-site disposal alternative provides the highest degree of effectiveness, is feasible to implement, and is also economically feasible.

Effectiveness

Removal and off-site disposal provides the highest degree of protection for human health and the environment by physically removing the materials from HPS. Removal and off-site disposal will also comply with chemical-, action-, and location-specific ARARs.

Implementability

This alternative does not have administrative constraints and has few technical constraints. Most of the radioactive contamination identified at HPS to date has been in localized areas. Surface scans performed in the past have found point-source anomalies and specific disposal areas containing radioactive contamination. Subsurface investigations have found concentrated areas where disposal of dials or other radioactive materials occurred. Physical removal is very feasible for these types of situations. If large quantities of radioactive materials are found, physical removal and off-site disposal may have significant technical constraints. Any areas found to contain large quantities of low-level radioactive waste will not be addressed by this AM, but will be evaluated further in the ongoing CERCLA process.

Cost

Unit costs for labor, mobilization, and site remediation are comparable with a standard soil removal and disposal project involving chemical contamination. The unit cost for disposal of radioactive materials is on average greater than the unit cost of chemical contamination in soils; however, the cost does not become prohibitive unless very large volumes of radioactive materials are removed and disposed. Further details regarding the unit costs for this alternative are provided in Section V.B.

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4. Engineering Evaluation and Cost Analysis

Since this is a time-critical removal action, an engineering evaluation and cost analysis is not applicable.

5. Applicable or Relevant and Appropriate Requirements

Section 300.415(j) of the NCP provides that removal actions must attain ARARs to the extent practicable, considering the exigencies of the situation.

Section 300.5 of the NCP defines <u>applicable requirements</u> as cleanup standards, standards of control, and other substantive environmental protection requirements, criteria or limitations promulgated under federal or state environmental or facility siting laws that specifically address a hazardous substance, pollutant, contaminant, remedial action, location or other circumstances at a CERCLA site.

Section 300.5 of the NCP defines <u>relevant and appropriate requirements</u> as cleanup standards, standards of control and other substantive requirements, criteria, or limitations promulgated under federal or state environmental or facility siting laws that, while not "applicable" to a hazardous substance, pollutant, or contaminant, remedial action, location, or other circumstances at a CERCLA site, address problems or situations sufficiently similar to those encountered at the CERCLA site and are well-suited to the particular site.

Because CERCLA on-site response actions do not require permitting, only substantive requirements are considered as possible ARARs. Administrative requirements such as approval of, or consultation with administrative bodies, issuance of permits, documentation, reporting, record keeping, and enforcement are not ARARs for CERCLA actions confined to the site.

Only those state standards that are identified by a state in a timely manner and are more stringent than federal requirements may be applicable or relevant and appropriate.

There are three types of ARARs: contaminant-specific, location-specific, and action-specific. The first type includes contaminant specific requirements. These ARARs set limits on concentrations of specific hazardous substances, contaminants, and pollutants in the environment. Examples of this type of ARAR are ambient water quality criteria and drinking water standards. The second type of ARAR includes location-specific requirements that set restrictions on certain types of activities based on site characteristics. These include restrictions on activities in wetlands, floodplains, and historic sites. The third type of ARAR includes action-specific requirements. These are technology-based restrictions which are triggered by the type of action under consideration. Examples of action-specific ARARs are Resource Conservation and Recovery Act regulations for waste treatment, storage, and disposal.

ARARs must be identified on a site-specific basis from information about specific chemicals at the site, specific features of the site location, and actions that are being considered as removal actions.

The ARARs used to prepare this AM are presented in Attachment A.

The cleanup goals presented in this AM were derived by considering the following:

- Soil cleanup goals: EPA decay-corrected PRGs (EPA 1991)
- Radium-226 contamination in soils: per agreement with EPA
- Radioactive contamination on structures: These limits are based on 25 millirem per year (mrem/y), using D&D Version 2 or Atomic Energy Commission's (AEC's) *Regulatory Guide* 1.86 (1974), whichever is lower.

- Radioactive contamination on surfaces designated as equipment or waste: These limits are based on AEC's *Regulatory Guide 1.86*. Limits for removable surface activity are 20 percent of these values.
- Application of soils, debris, and surface cleanup goals to sites: MARSSIM guidance (EPA and others 2001)
- Radioactive contamination in wastewater: A release criterion for water has been derived from *Radionuclides Notice of Data Availability Technical Document* (EPA, 2000) by comparing the limits from two criteria and using the most conservative limit.

The cleanup goals derived for the project are considered to be the most conservative available. For example, use of EPA decay-corrected PRGs for soil removal actions is more conservative than use of other federal ARARs listed in Attachment A.

6. Project Schedule

Individual removal action project schedules will be generated as each site is identified for removal of radiological materials. Based on previous removal actions for radiological materials, field events are expected to last from 1 to 4 months. Prior to commencing field work, detailed work plans and health and safety plans will be generated. Following field events, analytical reports, data validation reports, or summary reports will also be generated to summarize actions taken.

B. ESTIMATED COSTS

The Navy has made a present worth estimate of the removal action costs. The estimated costs include the direct and indirect capital costs. The items listed below are considered capital costs. They are based on a previous removal action, which removed and disposed of 17 cubic yards of contaminated soils. Costs for removal actions involving debris or surfaces will be comparable, if the quantity of radioactive materials disposed of is similar.

Estimated Costs – Typical 17-Cubic-Yard Soil Removal Action

Direct Capital Costs

Construction/Equipment/Materials: \$17,000 Soil excavation (\$75.00/cubic yard): \$1,275 Transport and disposal: \$21,250 Analytical (12 confirmation samples) \$5,400

Indirect Capital Costs

Work plans, engineering, etc. \$ 2,000

Soils Removal Action Total: \$46,925

VI. EXPECTED CHANGE IN THE SITUATION SHOULD ACTION BE DELAYED OR NOT TAKEN

If action should be delayed or not taken, exposure of human populations to low-level radiological materials may occur. Contamination may spread from HPS to nearby areas from wind erosion, surface water runoff, or other erosion mechanisms. Migration of radiological contamination could result in an

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increased health risk to local populations because of prolonged exposure to low-level radioactive materials.

Since the half-lives of radiological contaminants can range up to tens of thousands of years, the associated risk could be very long term, and migration over this time period may result in a greater volume of material to be remediated. This would also result in an increase in treatment or disposal costs.

VII. PUBLIC INVOLVEMENT

This document will be added to the administrative record (Attachment B) and will be made available for public review at the following locations:

San Francisco Public Library Government Documents 100 Larkin Street San Francisco, California 94102

Anna E. Waden Library 5075 Third Street San Francisco, California 94124

VIII. OUTSTANDING POLICY ISSUES

No outstanding policy issues exist for this removal action.

IX. RECOMMENDATION

To date, the Navy has not acquired evidence identifying other potentially responsible parties (PRP) at this site. However, information acquired in the future, including but not limited to, information acquired during the implementation of this removal action or future response actions at the site, could result in the identification of other PRPs.

This AM was prepared in accordance with current EPA and Navy guidance documents for TCRAs under CERCLA. The purpose of this AM is to identify and analyze removal actions to address localized radiological contamination in soils, debris/slag and buildings basewide.

Based on the analysis of the removal action alternatives completed in Section V.A.3, the recommended removal action is removal of radiological contamination from localized areas in soils, debris/slag, and buildings, followed by appropriate off-site disposal at a fully permitted disposal facility. This alternative will apply to localized areas throughout HPS, provide a high degree of protection for human health and the environment, does not have significant administrative or technical constraints, and is not cost prohibitive.

This decision document represents the selected removal action for HPS located in San Francisco, California, developed in accordance with CERCLA, as amended, and is not inconsistent with the NCP. This decision is based on the administrative record for the site (included in Attachment B).

Base Realignment and	
Closure Environmental	
Coordinator:	Keith S. Forman

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Date

REFERENCES

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- PRC. 1996. "Results of Subsurface Radiation Investigation in Parcels B and E, HPS, San Francisco, California." May 8.
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- TtEMI. 2000a. "Radiological Removal Action, Action Memorandum, HPS, San Francisco, California." August 17.
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- EPA. 1998. "A Fact Sheet on the Health Effects from Ionizing Radiation." EPA 402-F-98-010. May.
- EPA, U.S. Department of Energy, U.S. Department of Defense, and U.S. Nuclear Regulatory Commission. 2001. "Multi-Agency Radiation Survey and Site Investigation Manual." Revision 1. 65 Federal Register 62531. June 1.
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TABLES

TABLE 1
RELEASE CRITERIA

Radionuclide	Surfaces (dp	m/100 cm ²)	Soil ^c (p	oCi/g)	Water ^g
	Equipment, Waste ^a	Structures ^b	Outdoor Worker ^d	Residential ^d	(pCi/L)
Americium-241	100	23.9	5.67	1.87	15
Cesium-137	5,000	5,000	0.13 ^e	0.13°	119
Cobalt-60	5,000	5,000	0.0602	0.0361	100
Plutonium-239	100	24.7	14.0	2.59	15
Radium-226	100	100	2.0^{f}	2.0 ^f	5.0 ^h
Strontium-90	1,000	1,000	10.8	0.331	8
Thorium-232	1,000	6.49	19.0	3.1	15
Tritium-3	5,000	5,000	4.23	2.28	20,000
Uranium-235	5,000	86.6	0.417	0.205	30

Notes:

- ^a These limits are based on AEC *Regulatory Guide 1.86* (1974). Limits for removable surface activity are 20 percent of these values.
- These limits are based on 25 mrem/y, using D&D Version 2 or *Regulatory Guide 1.86*, whichever is lower.
- ^c EPA PRGs for two future-use scenarios.
- The on-site and off-site laboratory will ensure that the MDA meets the listed release criteria by increasing sample size or counting time as necessary. The MDA is defined as the lowest net response level, in counts, that can be seen with a fixed level of certainty, customarily 95 percent. The MDA is calculated per sample by considering background counts, amount of sample used, and counting time.
- ^e Decay-corrected PRG for industrial reuse provided by EPA Region IX.
- Limit is 1 pCi/g above background; not to exceed 2 pCi/g total, per agreement with EPA.
- Release criteria for water have been derived from *Radionuclides Notice of Data Availability Technical Document*, (EPA, 2000) by comparing the limits from two criteria and using the most conservative limit.
- h Limit is for total radium concentration.

AEC – Atomic Energy Commission

cm² – square centimeters

dpm – disintegrations per minute

EPA – U.S. Environmental Protection Agency

MDA – minimum detectable activity

mrem/y - millirem per year

pCi/g – picocurie per gram

pCi/L – picocurie per liter

PRG - Preliminary Remediation Goal

TABLE 2
ADDITIONAL POTENTIAL RADIONUCLIDES OF CONCERN

Radionuclide	Half-Life	Radiations						
Actinium-227	21.8 years	Alpha, beta, gamma						
Amercium-243	7,370 years	Alpha, gamma						
Barium-133	10.5 years	Beta, gamma						
Bismuth-207	32 years	Beta, gamma						
Carbon-214	5,715 years	Beta						
Chlorine-36	3.01 x 10 ⁵ years	Beta						
Curium-244	18.1 years	Alpha, gamma						
Europium-152	13.5 years	Beta, gamma						
Europium-154	8.6 years	Beta, gamma						
Gadolinium-152	1.1 x 10 ¹⁴ years	Alpha						
Indium-115	4.4 x 10 ¹⁴ years	Beta						
Potassium-40	1.27 x 10 ⁹ years	Beta, gamma						
Niobium-94	2 x 10 ⁴ years	Beta, gamma						
Nickel-63	100 years	Beta						
Neptunium-237	2.14 x 10 ⁶ years	Alpha, gamma						
Lead-210	22.6 years	Beta, gamma						
Plutonium-238	87.7 years	Alpha, gamma						
Technetium-97	2.6 x 10 ⁶ years	Beta, gamma						
Technetium-99	2.1 x 10 ⁵ years	Beta, gamma						
Titanium-44	67 years	Gamma						
Thallium-204	3.78 years	Beta						
Uranium-233	1.59 x 10 ⁵ years	Alpha, gamma						
Uranium-236	2.34 x 10 ⁷ years	Alpha, gamma						
Uranium-238	4.478 x 10 ⁹ years	Alpha, gamma						

TABLE 3
BUILDING/AREA ASSESSMENT AND CLASSIFICATION

Building No. or Area	Con	ıtamiı	nation	ı Poto	ential		Con	tam	inato	ed M	ledia			Pot	enti: Pa	al M thw		tion		
	Kn ow n- res tric ted Ac ces s	Kno wn- cont inue d Acc ess	Like ly	Unli kely	Unk now	e	Su bsu rfa ce Soi ls	e Wa		Air	uct	Dr ain age Sys te m	Su rfa ce Soi I	Su bs urf ace Soi l	Su rfa ce W ate r	Gr ou nd wa ter	Air	uct	Dr ain age Sys te m	
Parcel B																				
103				1		N	N	N	N	N	L	L	N	N	N	N	N	L	N	Review Final Status Survey Report
113				✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
113A				✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
114				✓		L	N	N	N	N	N	N	L	N	N	N	N	N	N	Scoping Survey
130				✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
140 and Discharge Channel				✓		N	N	N	N	N	L	L	N	N	N	N	N	L	L	Scoping Survey
142				✓		L	N	N	N	N	L	N	L	N	N	N	N	L	N	Scoping Survey
146			✓			N	N	N	N	N	L	N	N	N	N	N	N	L	N	Characterization Survey
157				✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Scoping Survey
IR-07				✓		L	L	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey

TABLE 3
BUILDING/AREA ASSESSMENT AND CLASSIFICATION

IR-18			✓	L	L	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey
Drydock 5			✓	N	N	N	N	N	L	L	N	N	N	N	N	L	L	Scoping Survey
Drydock 6			✓	N	N	N	N	N	L	L	N	N	N	N	N	L	L	Review Final Status Survey Report
Drydock 7			✓	N	N	N	N	N	L	L	N	N	N	N	N	L	L	Scoping Survey
Parcel C																		
203			✓	L	N	N	N	N	L	N	L	N	N	N	N	L	N	Scoping Survey
205 and Discharge Channel			✓	N	N	N	N	N	L	L	N	N	N	N	N	L	L	Scoping Survey
211	✓			N	N	N	N	N	M	L	N	N	N	N	N	L	L	Remediation and Final Status Survey
214			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
224		✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
241			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
253	1			N	N	N	N	N	Н	Н	N	N	N	N	N	M	M	Remediation and Final Status Survey
271			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
272			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
Drydock 2		✓		N	N	N	N	N	M	L	N	N	N	N	N	L	L	Review Final Status Survey Report
Drydock 3		✓		N	N	N	N	N	M	L	N	N	N	N	N	L	L	Review Final Status Survey Report
Drydock 4		✓		N	N	N	N	N	M	L	N	N	N	N	N	L	L	Review Final Status Survey Report

TABLE 3
BUILDING/AREA ASSESSMENT AND CLASSIFICATION

Parcel D																		
274			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
313 Site		✓		L	L	N	N	N	N	N	L	L	N	N	N	N	N	Review Final Status Survey Report
313A Site		✓		M	L	N	N	N	N	M	L	L	N	N	N	N	L	Review Final Status Survey Report
317 Site		✓		L	L	N	N	N	N	N	L	L	N	N	N	N	N	Review Final Status Survey Report
322 Site		✓		L	N	N	N	N	N	N	L	N	N	N	N	N	N	Review Final Status Survey Report
351		✓		N	N	N	N	N	M	L	N	N	N	N	N	L	L	Review Final Status Survey Report
351A	✓			M	N	N	N	N	M	M	M	N	N	N	N	L	L	Characterization Survey
364	✓			Н	M	N	N	N	Н	Н	M	L	N	N	N	M	M	Remediation and Final Status Survey
365			1	N	N	N	N	N	L	L	N	N	N	N	N	L	L	Review Final Status Survey Report
366/351B	✓			N	N	N	N	N	M	M	N	N	N	N	N	L	L	Remediation and Final Status Survey
383			1	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
408		✓		N	N	N	N	N	M	N	N	N	N	N	N	L	N	Scoping Survey
411			1	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
Gun Mole(Regunning)Pier		✓		L	L	N	N	N	L	N	L	L	N	N	N	L	N	Review Characterization Report
500			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Scoping Survey
503 Site		✓		N	L	N	N	N	N	L	N	L	N	N	N	N	L	Scoping Survey
Mahan Street-NRDL		✓		 M	M	N	N	N	N	N	L	L	N	N	N	N	N	Review Final Status Survey Report

TABLE 3
BUILDING/AREA ASSESSMENT AND CLASSIFICATION

813			✓	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Scoping Survey
819		✓		N	N	N	N	N	L	М	N	N	N	N	N	L	M	Scoping Survey
Parcel E																		
406		✓		N	N	N	N	N	M	N	N	N	N	N	N	L	N	Review Final Status Survey Report
414			1	N	N	N	N	N	L	N	N	N	N	N	N	L	N	Review Final Status Survey Report
500 Building Series		✓		M	Н	N	N	N	N	Н	L	M	N	N	N	N	Н	Scoping Survey
506 Site		✓		M	M	N	N	N	N	M	L	L	N	N	N	N	M	Scoping Survey
507 Site		1		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Characterization Survey
508 Site		✓		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Characterization Survey
509 Site		1		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Characterization Survey
510 Site		1		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Characterization Survey
510A Site		✓		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Scoping Survey
517 Site		✓		L	L	N	N	N	N	M	L	L	N	N	N	N	L	Characterization Survey
520 Site	✓			M	M	N	N	N	N	M	M	M	N	N	N	N	L	Characterization Survey
521			✓	L	N	N	N	N	L	N	N	N	N	N	N	N	N	Scoping Survey
529 Site	✓			M	M	N	N	N	M	Н	L	L	N	N	N	L	M	Scoping Survey
701 Site			✓	L	N	N	N	N	N	N	L	N	N	N	N	N	N	Review Final Status Survey Report
704 Area		✓		L	L	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey

TABLE 3
BUILDING/AREA ASSESSMENT AND CLASSIFICATION

704/Pens			✓	L	L	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey
707/Kennels	✓			L	L	N	N	N	L	M	L	L	N	N	N	L	M	Characterization Survey
707 B Site			✓	L	L	N	N	N	N	N	N	L	N	N	N	N	L	Characterization Survey (as part of 707 Triangle Area Survey)
707 C Site			✓	L	L	N	N	N	N	N	L	L	N	N	N	N	N	Characterization Survey as (part of 707 Triangle Area Survey)
707 Triangle Area	✓			L	Н	N	N	N	N	Н	L	M	N	N	N	N	M	Characterization Survey
708			✓	L	N	N	N	N	L	N	L	N	N	N	N	N	N	Review Final Status Survey Report
719 Site			✓	L	L	N	N	N	N	N	L	N	N	N	N	N	N	Scoping Survey
807 Site			✓	L	L	N	N	N	N	N	L	L	N	N	N	L	N	Scoping Survey
810	✓			M	N	N	N	N	M	N	L	N	N	N	N	L	N	Remediation and Scoping Survey
Shack 79 Site		✓		M	L	N	N	N	N	N	L	L	N	N	N	N	N	Final Status Survey
Shack 80 Site	✓			Н	M	N	N	N	N	N	M	L	N	N	N	N	N	Remediation and Final Status Survey
Experimental Shielding Range		✓		M	L	N	N	N	N	N	L	L	N	N	N	N	N	Review Final Status Survey Report
IR-01/21, Industrial Landfill	✓			Н	Н	N	N	N	N	N	M	M	N	N	N	N	N	Review Characterization Survey Report, Remediation, and Final Status Survey
IR-02, Bay Fill	✓			Н	Н	N	L	N	N	N	M	M	N	L	N	N	N	Characterization Survey
IR-03		✓		M	M	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey
IR-04	✓			Н	M	N	N	N	N	N	M	L	N	N	N	N	N	Characterization Survey

TABLE 3

BUILDING/AREA ASSESSMENT AND CLASSIFICATION

Former Salvage Yard			✓			M	M	N	N	N	N	N	L	L	N	N	N	N	N	Scoping Survey
Shoreline		✓				Н	M	L	N	N	N	N	M	M	L	N	N	N	N	Characterization Survey
ase-wide																				
Storm Drain lines		*				N	L	N	N	N	L	Н	N	L	N	N	N	L	M	Scoping/Characterization Surveys of systems associated with NRDL sites or sites associated with radium use
Sanitary Sewers		*				N	L	N	N	N	L	Н	N	L	N	N	N	L	M	Scoping/Characterization Survey of systems associated with NRDL sites or sites associated with radium use
Septic Systems			✓			N	M	N	N	N	N	Н	N	L	N	N	N	N	M	Scoping/Characterization Surveys of systems associated with NRDL buildings
Parcel F																				
Underwater Areas			✓			N	L	N	N	N	N	N	L	N	N	N	N	N	N	Scoping Surveys in areas of Operation CROSSROADS decontamination activities and site outfall discharge
All Ships' Berths				✓		L	L	N	N	N	L	N	N	L	N	N	N	L	N	Review Final Status Survey Report for completed berths; Scoping Survey on remainder
Off-site Facility																				
ICW 418				✓		N	N	N	N	N	L	N	N	N	N	N	N	L	N	Scoping Survey

Notes:

- H High = Evidence of contamination in the media or migration pathway has been identified.
- Low = The potential for contamination in the type of media or migration pathway is remote.
- M Moderate = The potential for contamination in the media or migration pathway exists, although the extent has not been fully assessed.

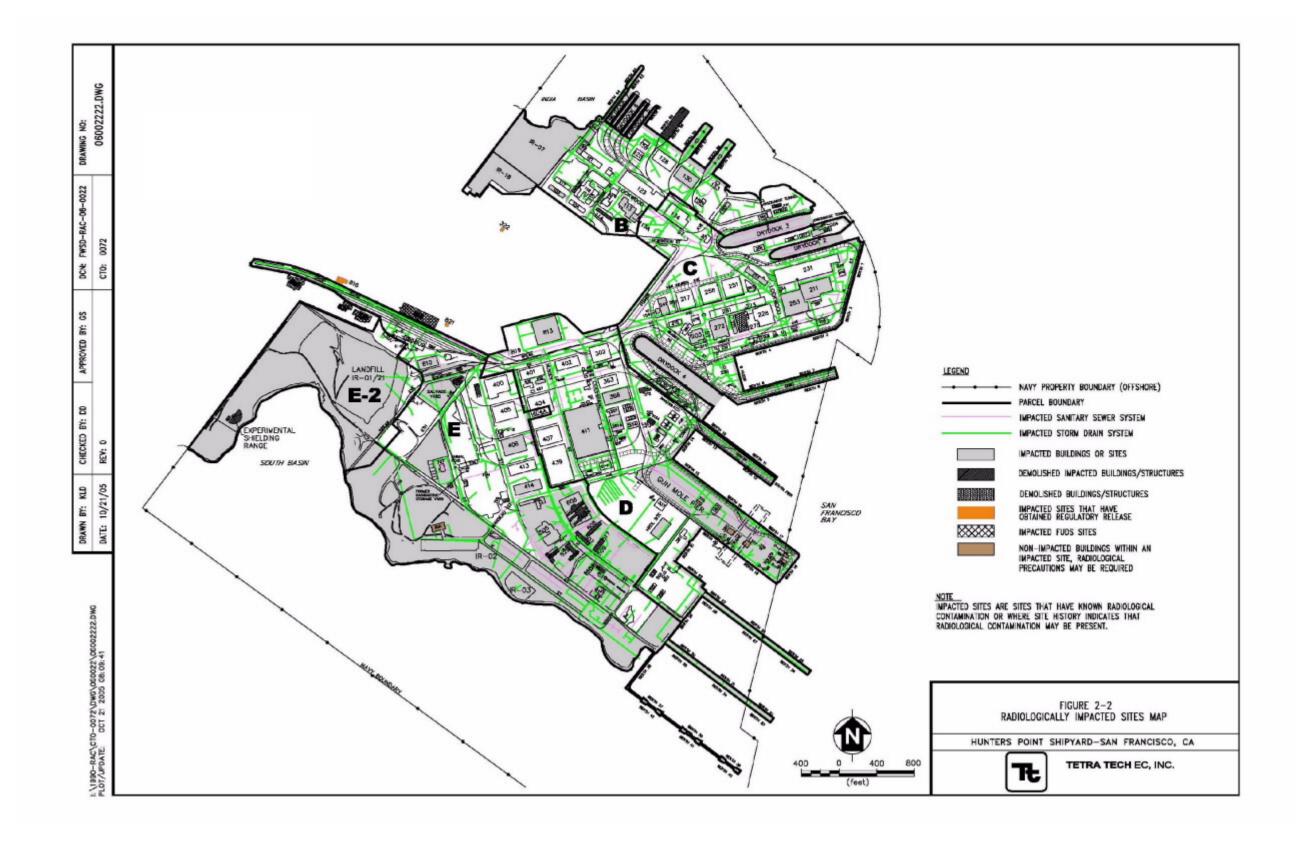
TABLE 3

BUILDING/AREA ASSESSMENT AND CLASSIFICATION

None = Evidence of contamination in the specific media or migration pathway has not been found, or known contamination has been removed, and surveys indicate that the media or migration pathway meets today's release criteria.

NRDL – Navy Radiological Defense Laboratory

FIGURES



Regulation	Requirement	Citation ^b	ARAR Determination	Comments		
Chemical-specific ^a AR	Chemical-specific ^a ARAR					
Health and Environmental Standards for Drinking Water	MCLs for radionuclides • Combined radium-226 and radium-228 – 5 pCi/L • Gross alpha (including radium-226 but excluding radon and uranium) – 15 pCi/L • Tritium – 20,000 pCi/L • Strontium-90 – 8 p/Ci/L • Beta and photon – 4 mrem/yr • Uranium – 30 μg/L	40 CFR §141.66	Applicable	This requirement is applicable to the wastewater limits established for the site.		
Radiological Criteria for Unrestricted Use at Closing NRC Licensed Facilities	A site will be considered acceptable for unrestricted use if the residual radioactivity that is distinguishable from background radiation results in TEDE to an average member of the critical group that does not exceed 25 mrem/yr, including that from groundwater sources of drinking water, and that the residual radioactivity has been reduced to ALARA.	10 CFR § 20.1402	Relevant and Appropriate	The DON would apply this requirement, with the exception that 15 mrem/y TEDE is substituted for 25 mrem, as ALARA.		
Radiological Criteria for License Termination Under Restricted Conditions	As a condition for license termination with restricted site use, the licensee must demonstrate that further reductions in residual radioactivity necessary to comply with the provisions of 10 USC § 20.1402 would result in net public or environmental harm or were not being made because the residual levels associated with restricted conditions are ALARA.	10 CFR § 20.1403(a)	Relevant and Appropriate	Potentially relevant and appropriate for a restricted land use scenario since radioactive materials may be left on site at fill areas.		
Radiological Criteria for License Termination Under Restricted Conditions	As a condition for license termination with restricted site use, the licensee must make provisions for legally enforceable institutional controls that provide reasonable assurance that the TEDE from residual radioactivity distinguishable from background to the average member of the critical group will not exceed 25 mrem/yr.	10 CFR § 20.1403(b)	Applicable	Restricted land use scenarios may be used in areas that are associated with fill and surveys will not be performed at depths greater than 1 foot below ground surface. The 25 mrem/yr will still apply to the surface dose.		

Regulation	Requirement	Citation ^b	ARAR Determination	Comments		
Chemical-specific ^a ARAR (Continued)						
Alternate Radiological Criteria for License Termination	Alternate criteria are allowed for license termination as long as assurance is provided that public health and safety would continue to be protected and that it is unlikely that the dose from all manmade sources combined, other than medical, would be more than the 100 mrem/yr limit of subpt. D, by submitting an analysis of possible sources of exposure; to the extent practical restrictions on site use are employed according to the provisions of § 20.1403 in minimizing exposures at the site; and doses are reduced to ALARA levels, taking into consideration any detriments such as traffic accidents expected to potentially result from decontamination and waste disposal.	10 CFR §20.1404(a)(1) - (a)(3)	Applicable	Sites will be dose modeled to show that residual radioactivity present does not exceed 25 mrem/yr. Therefore, members of the public are not expected to receive more than 100 mrem/yr.		
Dose Limits for Individual Members of the Public	Requires that the TEDE to individual members of public not exceed 0.1 rem from licensed operation: construction, operation, and decommissioning of commercial reactors and fuel cycle facilities; possession, use, processing, exporting, and certain aspects of transporting nuclear materials and waste; and siting, design, construction, operations, and closure of waste disposal sites.	10 CFR §20.1301(a)(1)	Applicable	This requirement is a health-based standard that is applicable for exposure to members of the public during removal actions.		
ALIs and DACs of Radionuclides for Occupational Exposure	Establishes limits for effluent releases to unrestricted area particularly in the implementation of the provisions of §20.1302, which implement the radiation dose limits for the public as listed in 10 CFR §20.1301		Applicable	This requirement is applicable to all removal actions performed as gaseous emissions of radionuclides may occur and is addressed by 10 CFR §20.1301.		
Location-specific ARA	Location-specific ARAR					
Federal Coastal Zone Management Act	This act specifies that federal actions that affect the coastal zone must be consistent with the policies of the San Francisco Bay Conservation and Development Commission's federally approved coastal management program.	16 USC 1456(c)(1)(A)	Applicable	This requirement is applicable to all removal actions performed in close proximity to San Francisco Bay.		

Regulation	Requirement	Citation ^b	ARAR Determination	Comments		
Action-specific ARAR						
Storage and Control of Licensed Material	The licensee shall secure from unauthorized removal or access licensed materials that are stored in controlled or unrestricted areas.	10 CFR §20.1801	Relevant and Appropriate	Potentially relevant and appropriate for a restricted land use scenario since waste will be left on site.		
	The licensee shall control and maintain constant surveillance of licensed material that is in a controlled or unrestricted area and that is not in storage.	10 CFR §20.1802	Relevant and Appropriate	Potentially relevant and appropriate for sites where radioactive material may remain on site if the site can meet the criteria.		
Protection of the General Population from Releases of Radioactivity	ation Concentrations of radioactive material that may be released to the		Applicable	Applicable for sites where radioactive materials may remain on site if the site can meet the criteria.		
Protection of the Individuals from Inadvertent Intrusion	Design, operation, and closure of the land disposal facility must ensure protection of any individual inadvertently intruding into the disposal site and occupying the site or contacting the waste at any time after active institutional controls over the disposal site are removed.	10 CFR §61.42	Relevant and Appropriate	Potentially relevant and appropriate for sites where radioactive waste may remain on site if the site can meet the criteria.		
Protection of Individuals During Operation	Every reasonable effort shall be made to maintain radiation exposures ALARA.	10 CFR §61.43	Applicable	Potentially relevant and appropriate for sites where radioactive waste will remain on site.		
Stability of the Disposal Site After Closure	osal Site After closed to achieve long-term stability of the disposal site and to		Relevant and Appropriate	Potentially relevant and appropriate for a site with radionuclides.		
Waste Disposal	A licensee shall dispose of licensed material only by: transfer to an authorized recipient, by decay in storage; or by release in effluents within the limits in § 20.1301; or as authorized under §§ 20.2002, 20.2003, 20.2004, or 20.2005.		Relevant and Appropriate	Potentially relevant and appropriate for a site with radionuclides.		

POTENTIAL FEDERAL AND STATE APPLICABLE OR RELEVANT AND APPROPRIATE REQUIREMENTS AND TO-BE-CONSIDERED CRITERIA FOR POTENTIALLY CONTAMINATED SITES AT HUNTERS POINT SHIPYARD

Release into Sanitary Sewage if each of the readily solubl into the sewer of water relea 3 of appendix released, the landle 3 of appendix released, the landle 3 of appendix sanitary sewer concentration sewer by the cappendix B to radionuclide receded unity; radioactive material is in Waste oils that	nay discharge licensed material into sanitary sewerage e following conditions is satisfied: the material is ole in water; and the quantity that the licensee releases er in 1 month divided by the average monthly volume eased does not exceed the concentration listed in Table x B to pt. 20; and if more than one radionuclide is a licensee shall determine the fraction of the limit in	10 CFR §20.2003	Relevant and Appropriate	The Navy would apply this requirement, with the exception
Release into Sanitary Sewage if each of the readily solubl into the sewer of water releas 3 of appendix released, the I Table 3 of appendix released, the I Table 3 of appendix sanitary sewer concentration sewer by the appendix B to radionuclide reced unity; radioactive mexceed unity; radioactive mexceed unity. Waste Treatment or Disposal by Incineration A licensee material is in Waste oils that	e following conditions is satisfied: the material is ble in water; and the quantity that the licensee releases er in 1 month divided by the average monthly volume eased does not exceed the concentration listed in Table x B to pt. 20; and if more than one radionuclide is	10 CFR §20.2003		
Disposal by Incineration only: as authorized material is in Waste oils that	ppendix B to pt. 20 represented by discharges into erage by dividing the actual monthly average n of each radionuclide released by the licensee into the concentration of that radionuclide listed in Table 3 of to pt. 20; and the sum of the fractions for each required by paragraph (a)(3)(i) of this section does not r; and the total quantity of licensed and other material that the licensee releases into the sanitary stem in a year does not exceed 5 Ci (185 GBq) of 1 Ci (37 GBq) of carbon-14, and 1 Ci (37 GBq) of all ctive materials combined.			that waste water discharged to the sanitary sewer system will meet the release criteria established in Table 5-1, which is more conservative than the values listed in this ARAR.
radioactive ef such incinerat	hay treat or dispose of licensed material by incineration norized by paragraph (b) of this section; or if the n a form and concentration specified in § 20.2005. The have been radioactively contaminated in the course tion or maintenance of a nuclear power reactor may be not the site where generated provided that the total effluents from the facility, including the effluents from ation, conform to the requirements of appendix I to part apter and the effluent release limits contained in	10 CFR §20.2004(a)	Relevant and Appropriate	The DON would apply this requirement, with the exception that waste will not be treated or incinerated on-site.

Radiological Removal Action, Hunters Point Shipyard DCN: FWSD-RAC-06-0022 Revision 1, December 14, 2005

POTENTIAL FEDERAL AND STATE APPLICABLE OR RELEVANT AND APPROPRIATE REQUIREMENTS AND TO-BE-CONSIDERED CRITERIA FOR POTENTIALLY CONTAMINATED SITES AT HUNTERS POINT SHIPYARD

Notes:

- ^a Many potential action-specific ARARs contain chemical-specific limitations and are addressed in the action-specific ARAR tables
- b Only the substantive provisions of the requirements cited in this table are potential ARARs

Abbreviations and Acronyms:

ALARA – as low as reasonably achievable

ALI – Annual Limit of Intake

ARAR – applicable or relevant and appropriate requirement

CFR – Code of Federal Regulations

Ci – curie

DAC – derived airborne concentration

DoN – Department of the Navy

GBq – gigabecquerel

LLRW – low-level radioactive waste

MCL - Maximum Contaminant Level

mrem – millirem

mrem/yr (millirem/year) – millirem per year

NRC - Nuclear Regulatory Commission

pCi/L – picocurie per Liter

pt. – part

subpt. - subpart

TEDE – total effective dose equivalent

μg/L – microgram per liter

USC - United States Code

APPENDIX B ADMINISTRATIVE RECORD INDEX

LIST OF DOCUMENTS IN ADMINISTRATIVE RECORD

Document Date	Document Type	Classification	Author Affiliation	Title or Subject
11/03/92	Report	AR	PRC	Surface Confirmation Radiation Survey (Phase I Investigation)
05/08/96	Report	AR	PRC	Results of Subsurface Radiation Investigation in Parcels B and E, HPS, San Francisco, California (Phase II Investigation)
10/27/97	Report	AR	TtEMI	Draft Final Parcel E RI Report, HPS, San Francisco, California (Phase I-III Investigation Summary)
05/15/00	Report	AR	TtEMI	Draft Phase IV Radiation Investigation Report, HPS, San Francisco, California
08/17/00	Report	AR	TtEMI	Radiological Removal Action, Action Memorandum, HPS, San Francisco, California
11/19/01	Report	AR	DON	Basewide Radiological Action, Action Memorandum, HPS, San Francisco, California
8/31/04	Report	AR	DON	Final Historical Radiological Assessment Volume II, History of the Use of General Radioactive Materials, 1939-2003, HPS, San Francisco, California

Notes:

AR – Administrative Record

DON – Department of the Navy HPS – Hunters Point Shipyard

PRC – PRC Environmental Management, Inc.

RI – Remedial Investigation

TtEMI – Tetra Tech EM, Inc.